

# RFP Performance Enhancement Through MHD Mode Control

John Sarff & MST Team

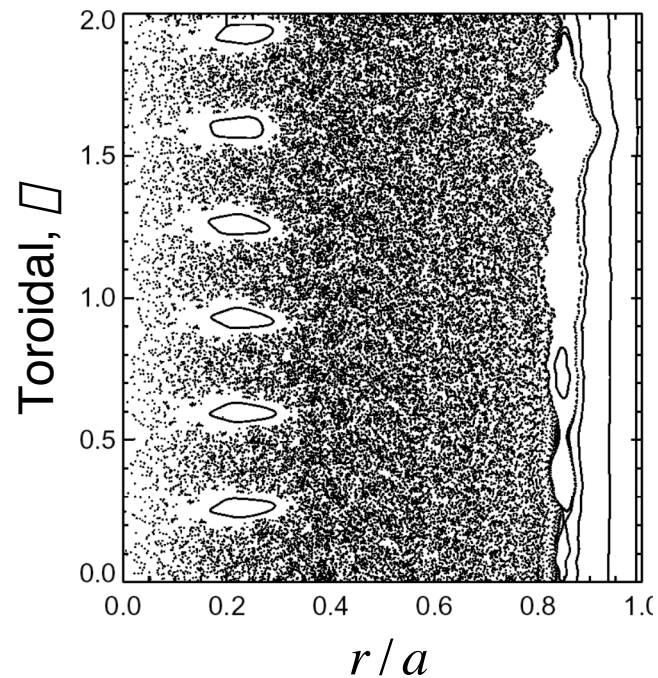


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# Standard RFP confinement is limited by MHD tearing.

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- Steady toroidal induction in low BT geometry drives current profile peaking:
  - $m = 1,0$  resistive MHD tearing unstable
  - Instability saturates by  $J(r)$  self-flattening through dynamo relaxation
- Performance characteristically high beta  $\beta < 20\%$ , but relatively poor confinement  $\tau_E > 50$  m<sup>2</sup>/s.



multiple tearing modes



global stochasticity

# Two paths to improved RFP confinement.

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- **Optimized magnetic relaxation:**
  - a) Induce and control “single-helicity” dynamo (*P. Martin’s talk*)
  - b) Possible favorable Lundquist number scaling for “multiple-helicity” relaxation (all tearing modes decrease together at higher temperature)
- **Current profile control for tearing stability:** (*this talk*)
  - Minimize magnetic relaxation
  - All tearing modes vanish (in principle)

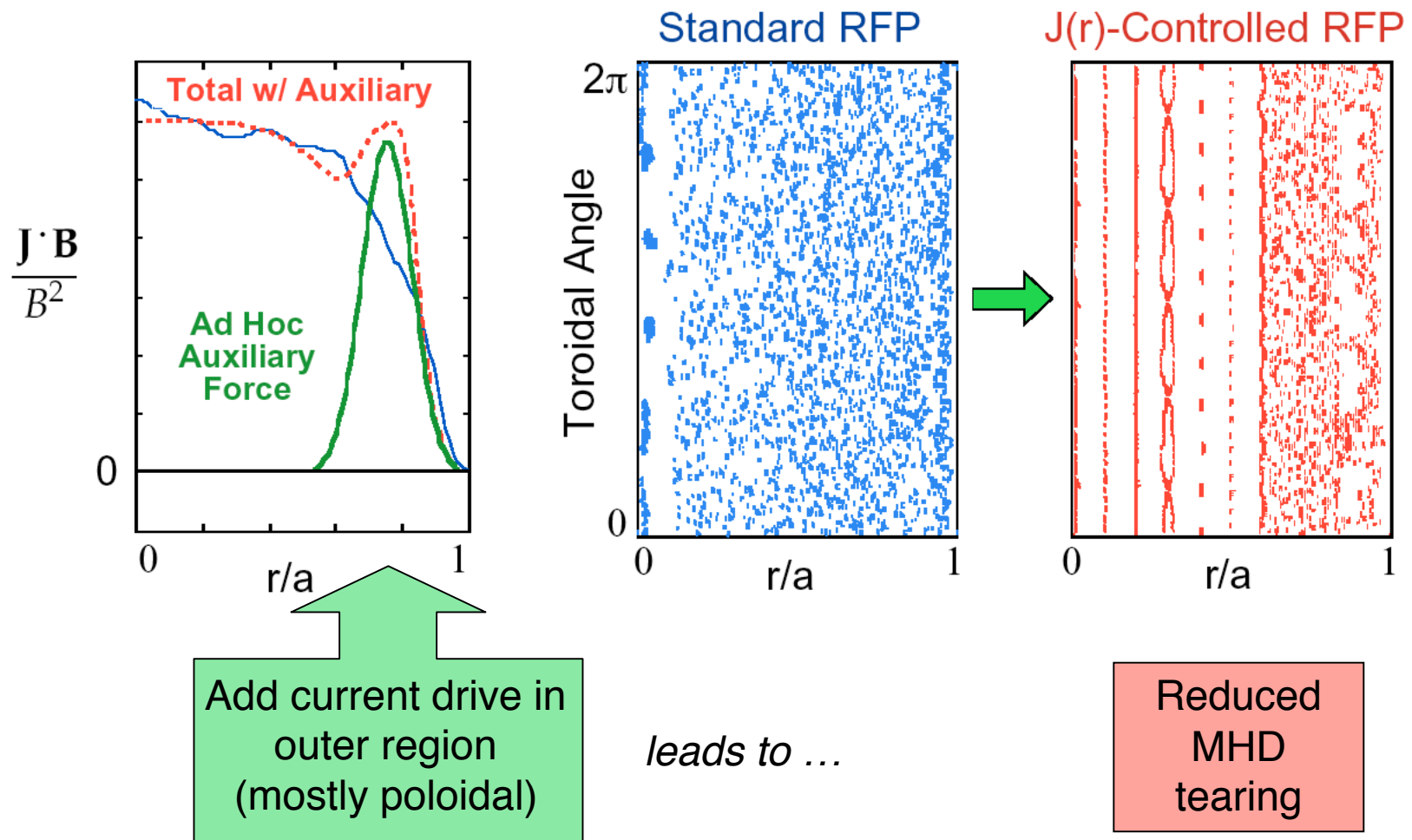
Goal for both paths is reduced magnetic stochasticity.

# Outline.

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- 
- MHD foundation for an RFP without dynamo
  - Improved confinement in MST:
    - $J(r)$  modification via inductive current drive
    - Tokamak-like confinement at high beta, low BT in MST
  - Broadband mode reduction key to reduced transport in the RFP
  - Control tools in development:
    - RF current drive (& heating)
    - Neutral beam heating

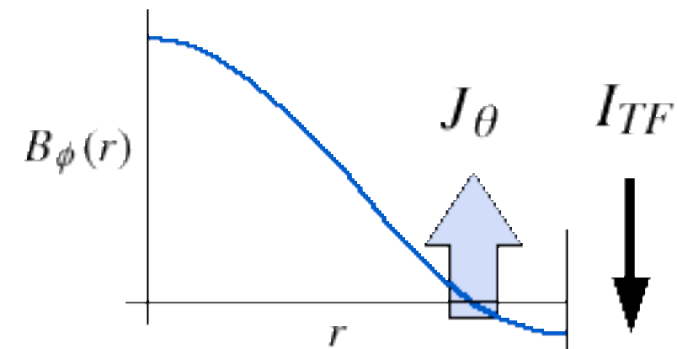
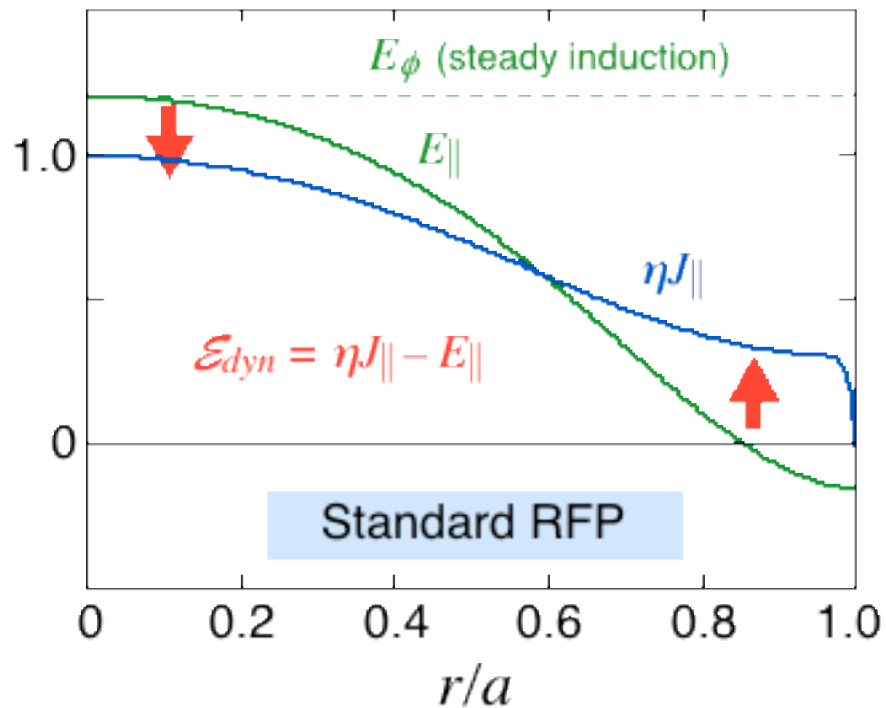
# MHD computation predicts current drive in outer region reduces tearing and magnetic stochasticity.



# Steady toroidal induction $E_\phi$ cannot support outer-region $J_\parallel$ .

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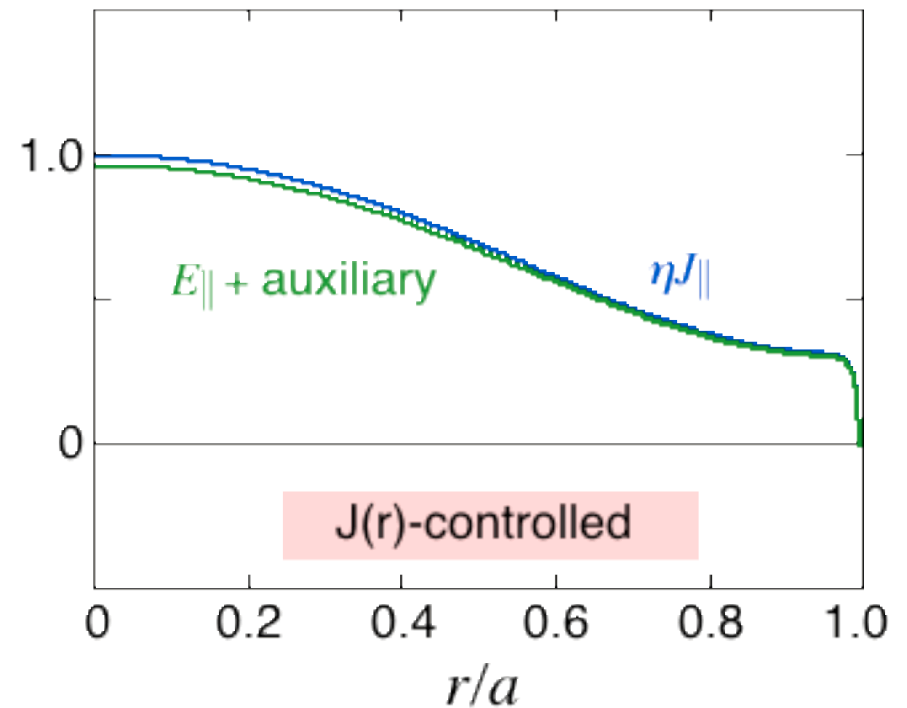
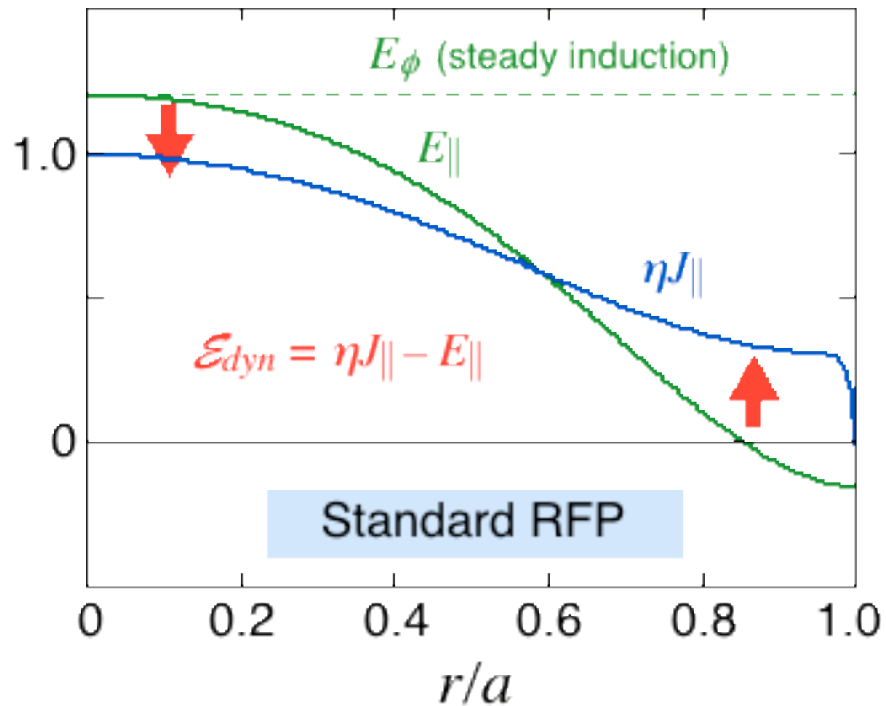
- Standard RFP requires dynamo current drive.



# Auxiliary current drive replaces dynamo sustainment of the required poloidal current.

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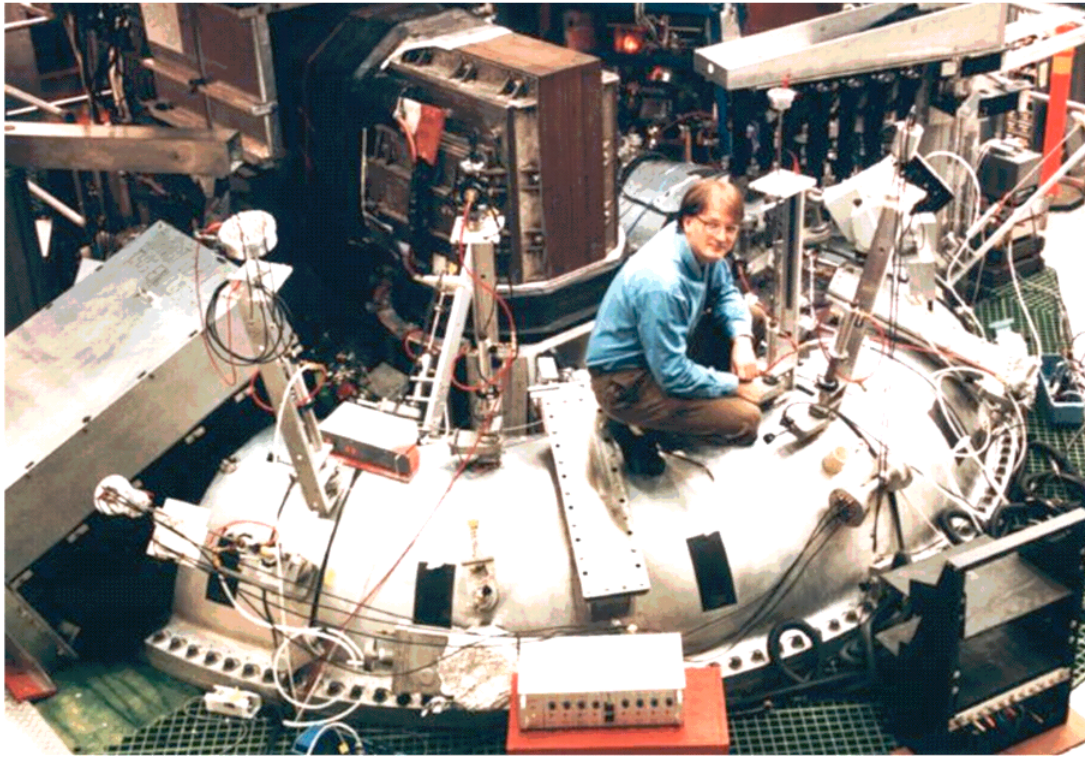
- Current drive “aligned” to current profile.



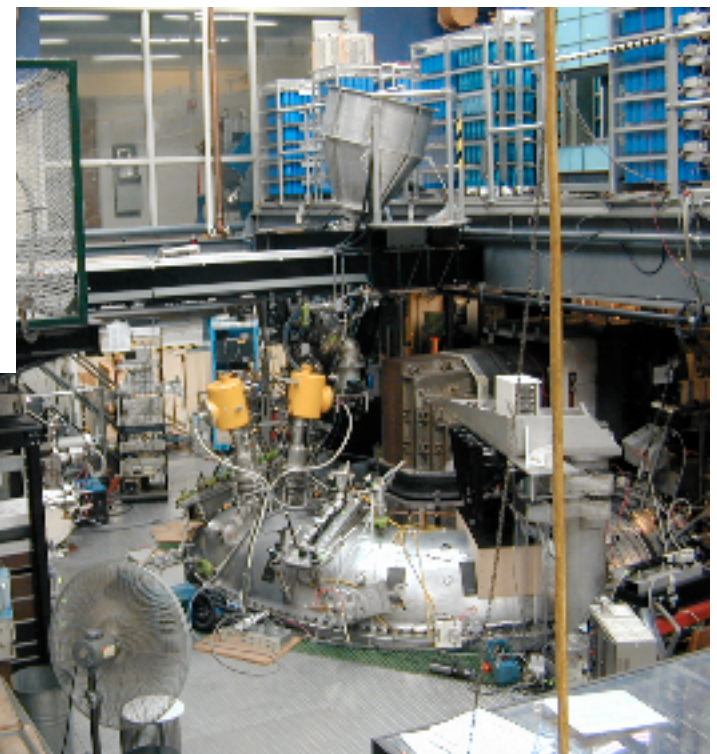


# Madison Symmetric Torus (MST)

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$a = 0.5 \text{ m}$   
 $R = 1.5 \text{ m}$   
 $I \leq 0.5 \text{ MA}$

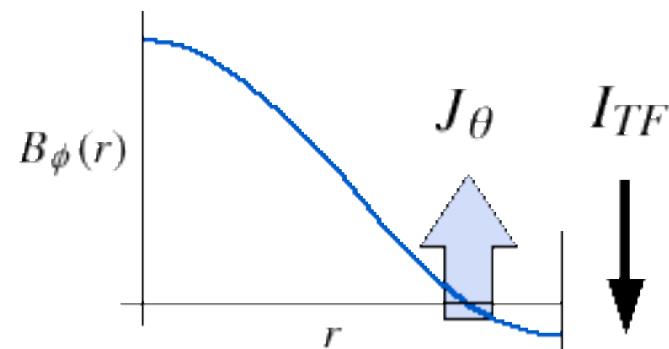
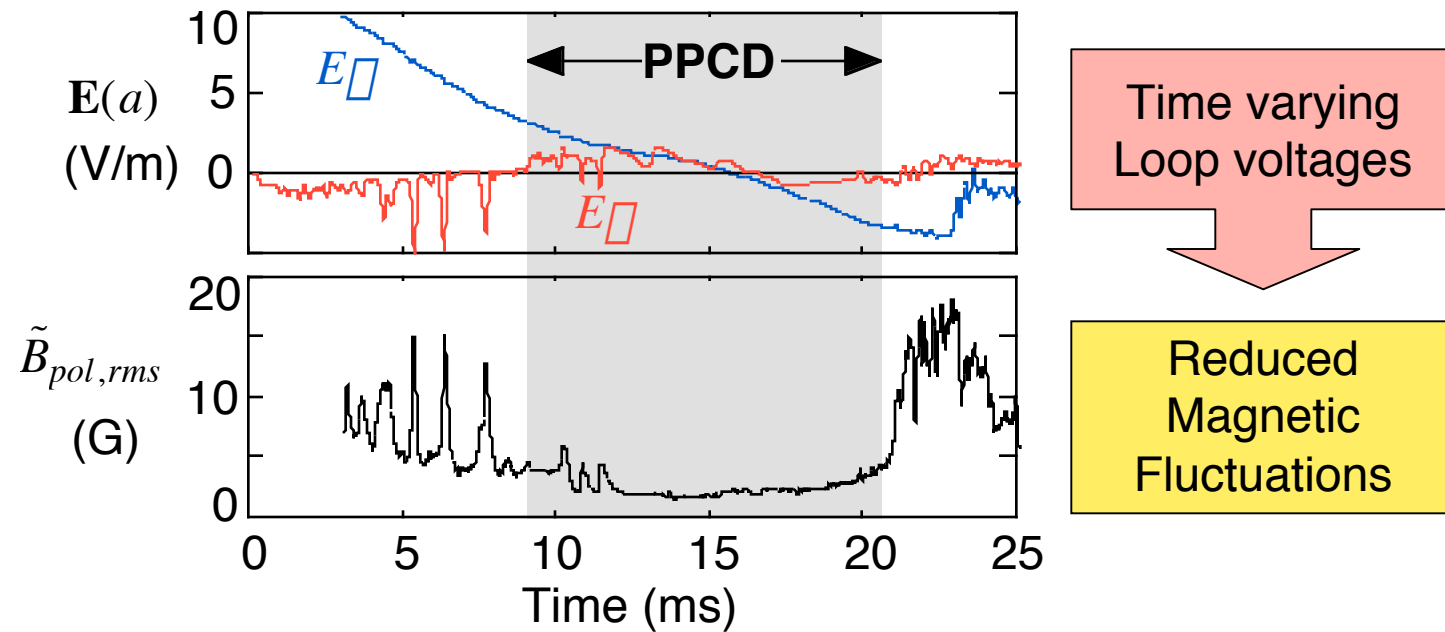




# Programmed inductive loop voltages provide current drive targeted to edge region.

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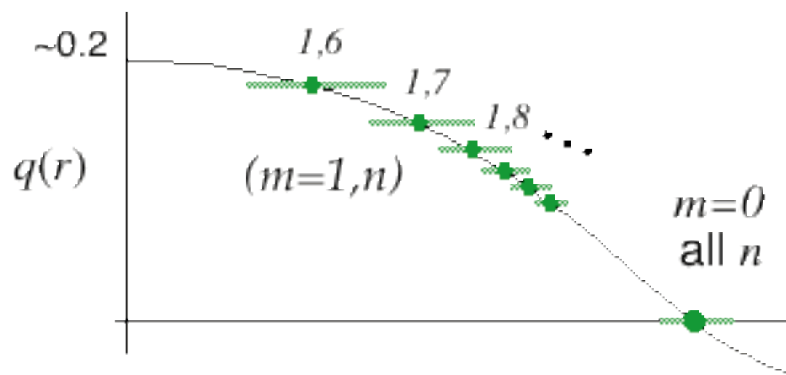
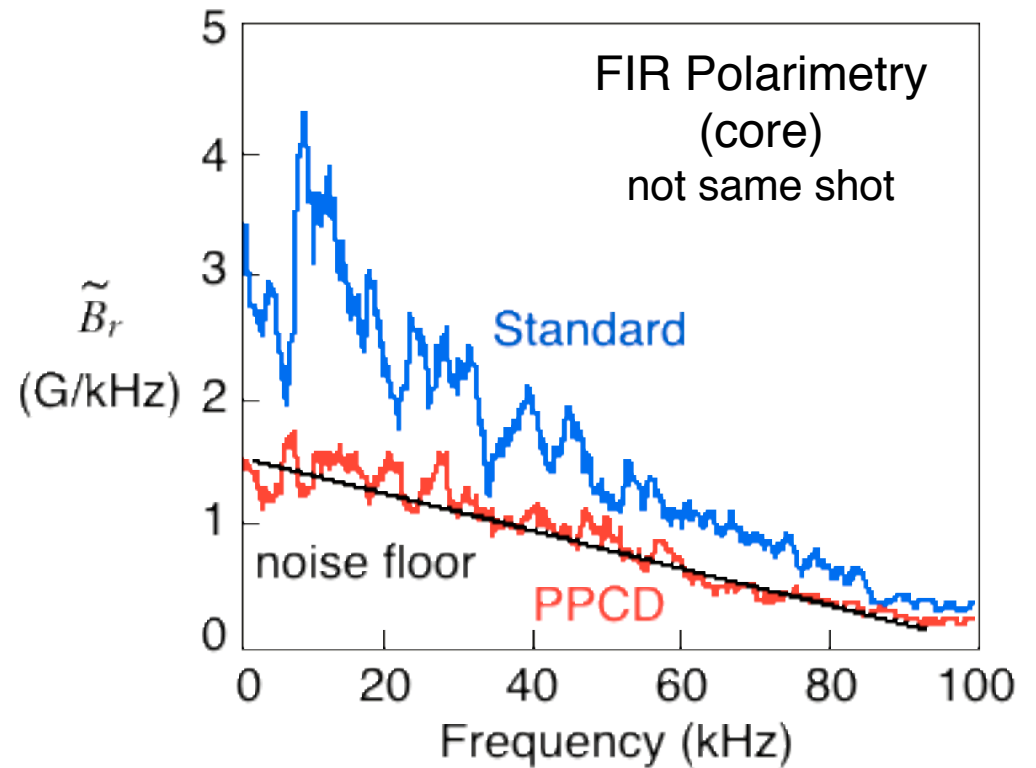
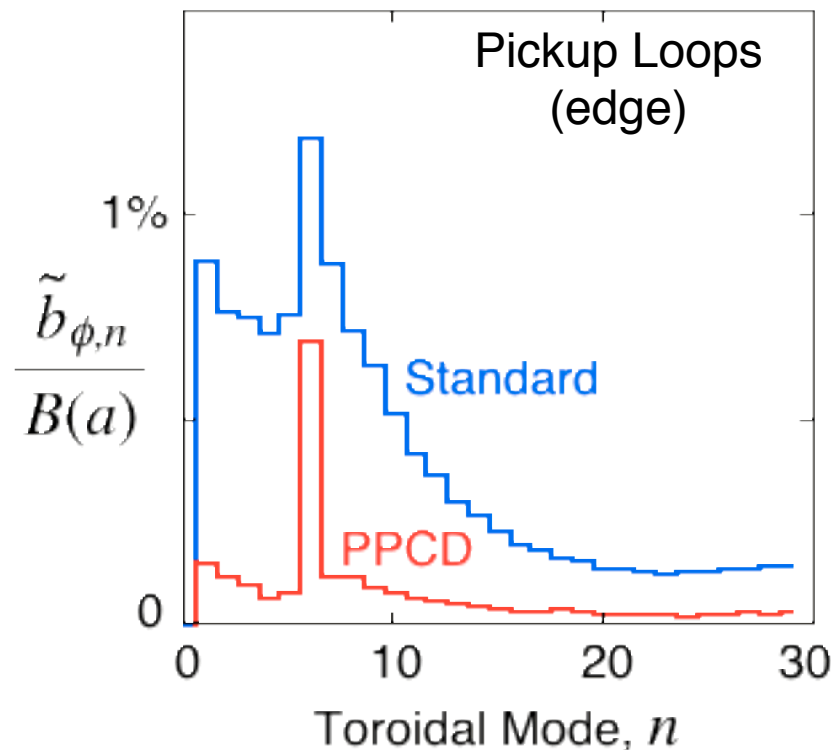
“PPCD” – Pulsed Poloidal (or Parallel) Current Drive



# Edge current drive simultaneously controls many modes.

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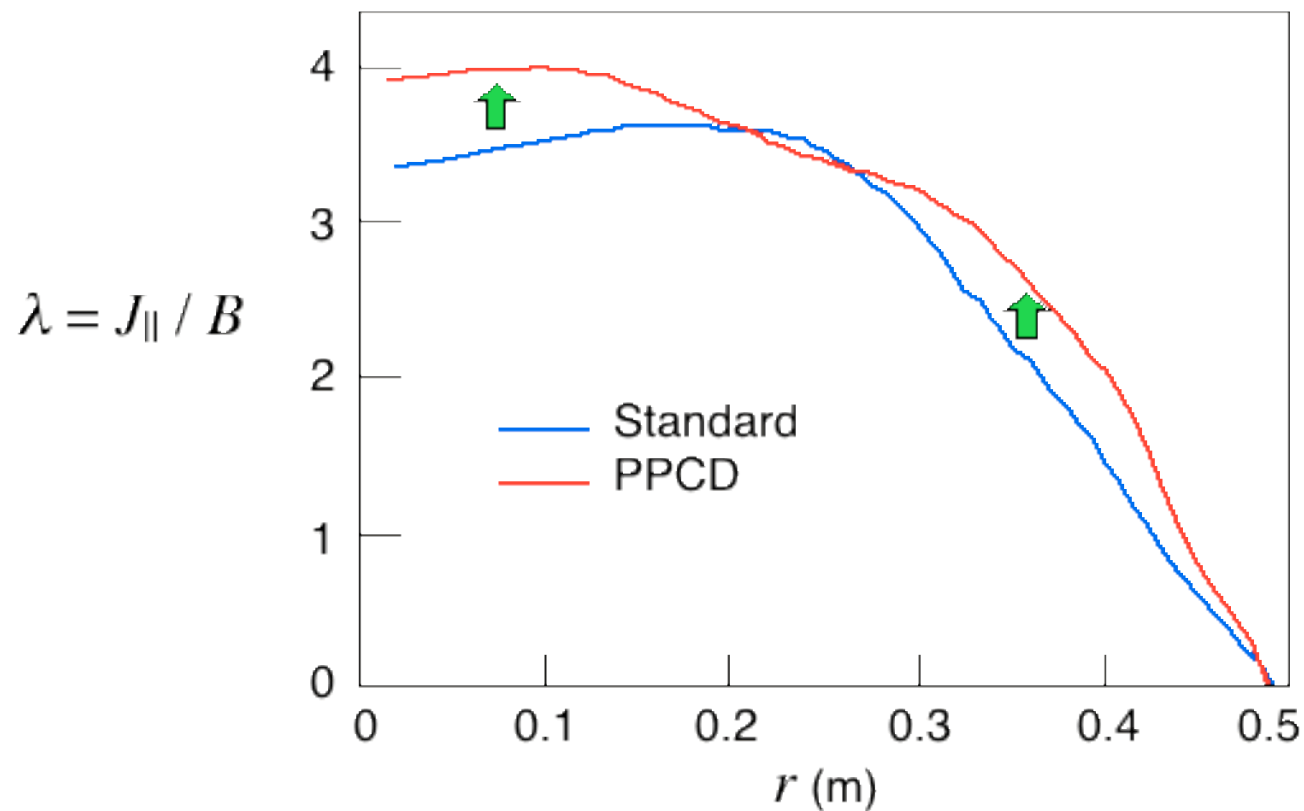
- Broadband mode suppression during PPCD.



## $J/B$ larger in outer region, as intended.

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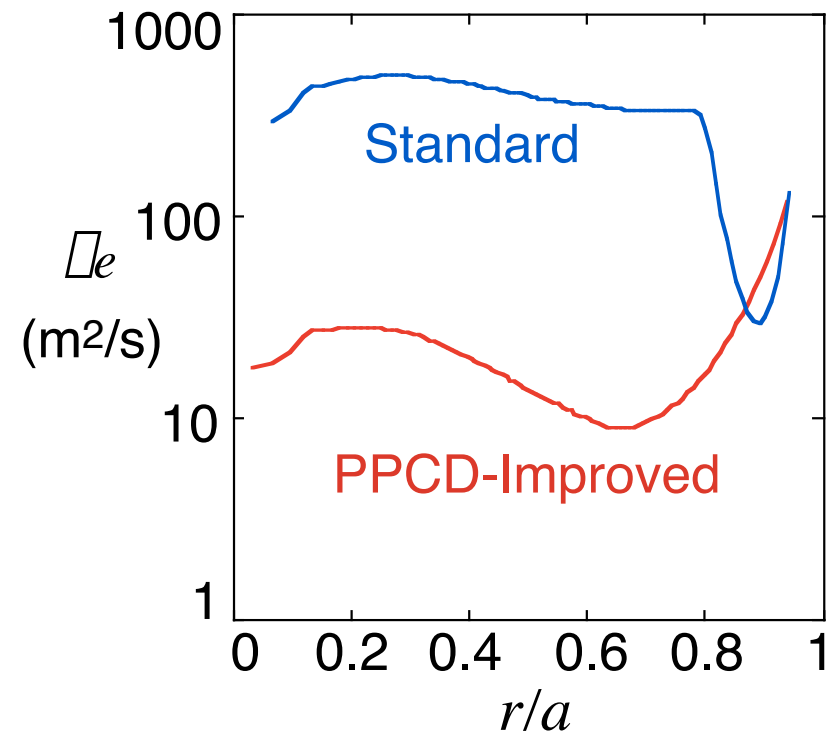
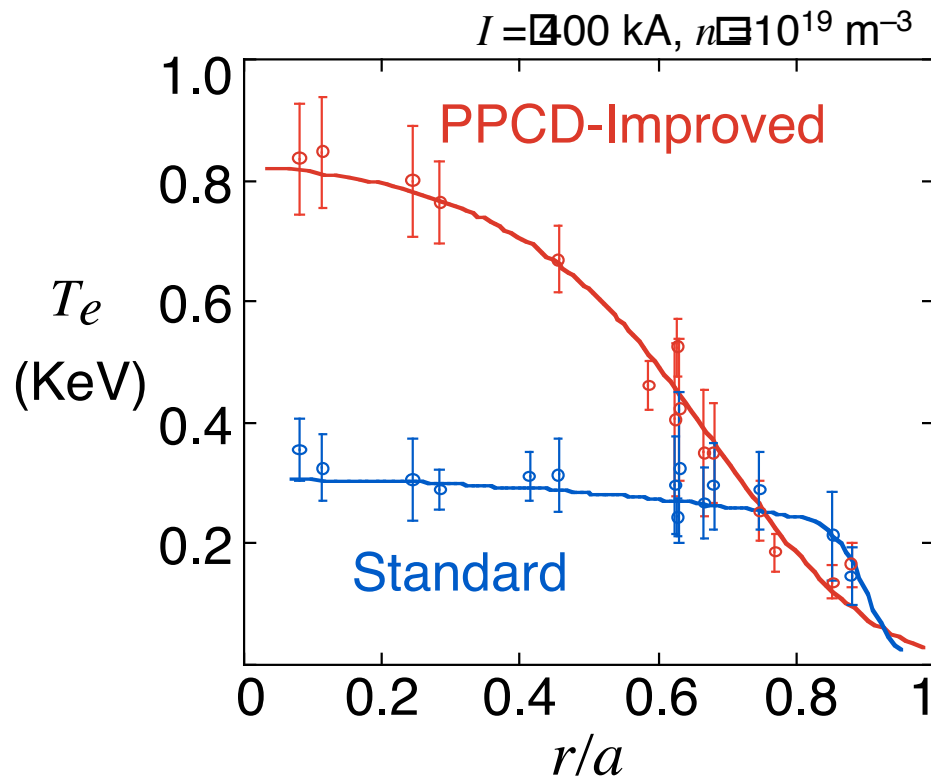
- Toroidal equilibrium reconstruction using:
  - 11-chords FIR polarimetry (UCLA collab.)
  - $B(0)$  from MSE with DNB (BINP collab.)
  - Conventional edge magnetics



# Temperature profile peaks, $\chi_e$ greatly reduced.

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- Electrons are hotter with **reduced** Ohmic heating, gradient extends into core.
- 100-fold increase in hard x-ray bremsstrahlung implies confined fast electrons.



MST maximum  $T_e(0) = 1.3 \text{ keV}$   
for  $I = 500 \text{ kA}$  PPCD

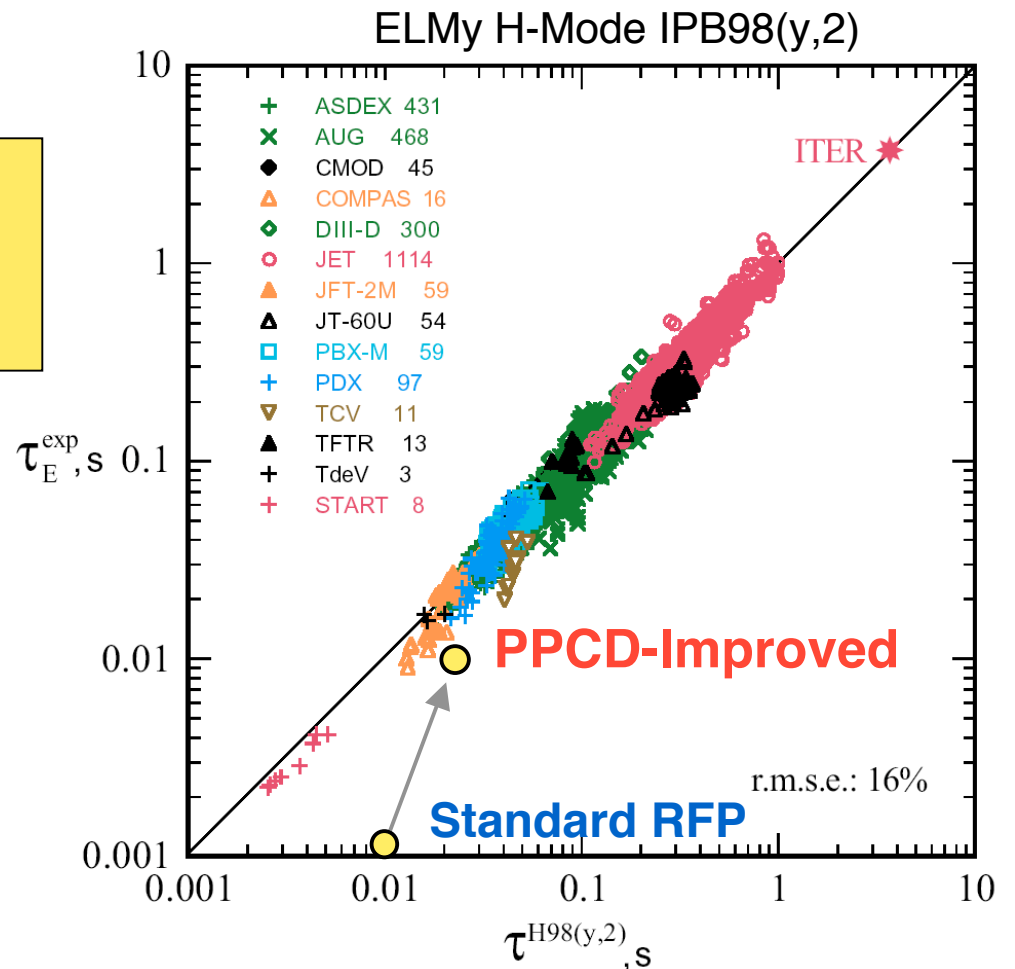
# PPCD confinement comparable to same-current tokamak, but with 10X smaller $B(a)$ in the RFP.

- Compare  $\tau_E \approx 10$  ms for 200 kA PPCD with tokamak  $\tau_E$  empirical scaling:
  - use “engineering” formulas with MST’s  $I, n, P$ , size & shape, but tokamak  $B_{T(a)} = 1.0$  T (corresponding to  $q_a = 4$ ).

Same-current RFP:

- $B_{T(a)} = 0.04$  T  $\square$  **20X smaller**
- $B(a) = 0.09$  T  $\square$  **10X smaller**


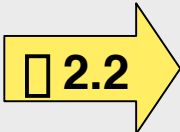
Comparable  $\tau_E$  does not imply tokamak empirical scaling applies to the RFP.



# PPCD doubles the already high beta value in RFP.

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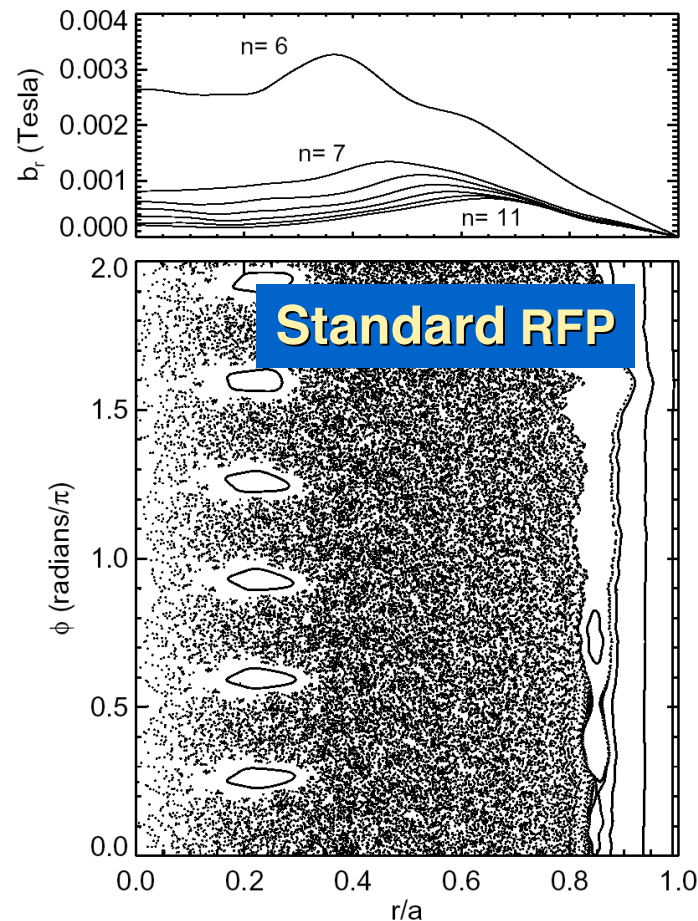
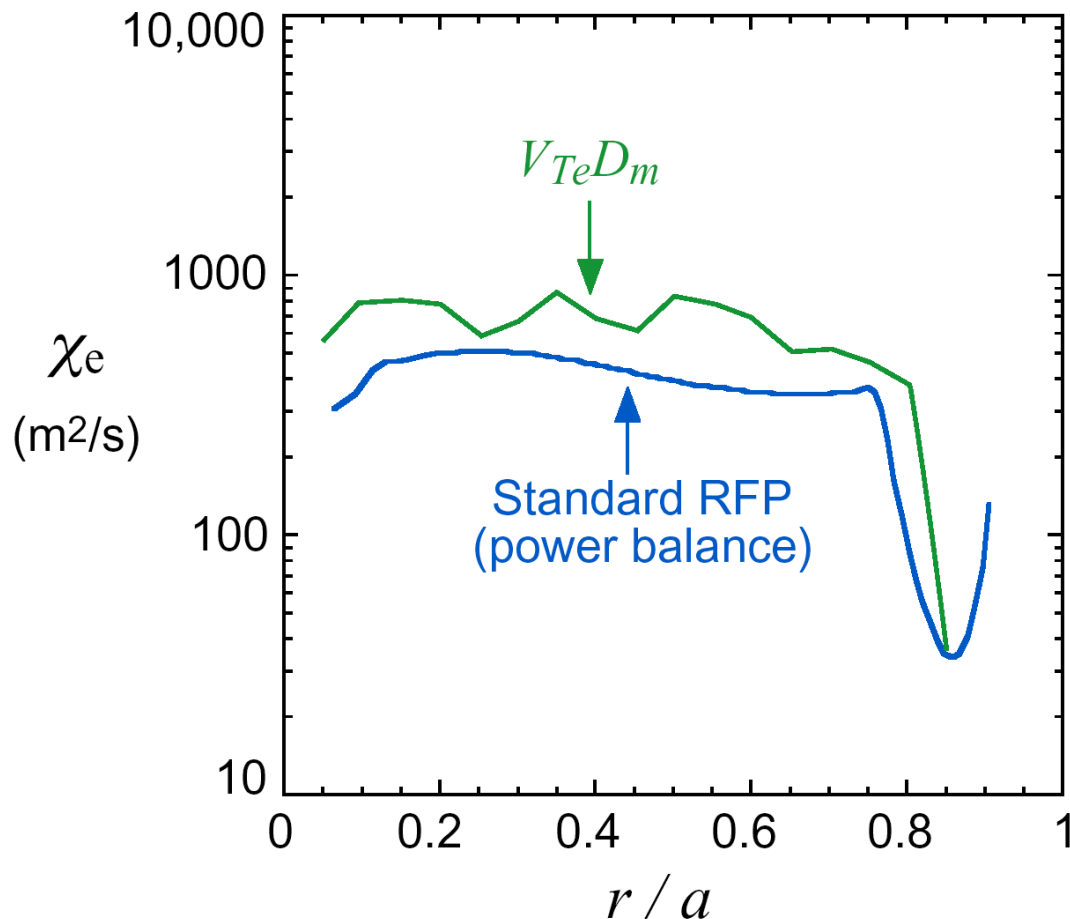
- Experimental  $\beta$ -limit unknown in the RFP (possibly pressure-tearing or g-mode??)

	Total Beta $\beta_{total} = \beta_p / B^2(a)$	
	Standard	PPCD-Improved
200 kA	9%	15% 
400 kA	5%	11% 

Toroidal beta *decreases* during PPCD to  $\beta_T \sim 80\%$  (from  $\gg 100\%$ )

# Bulk electron heat transport in standard plasmas agrees with stochastic diffusion expectations.

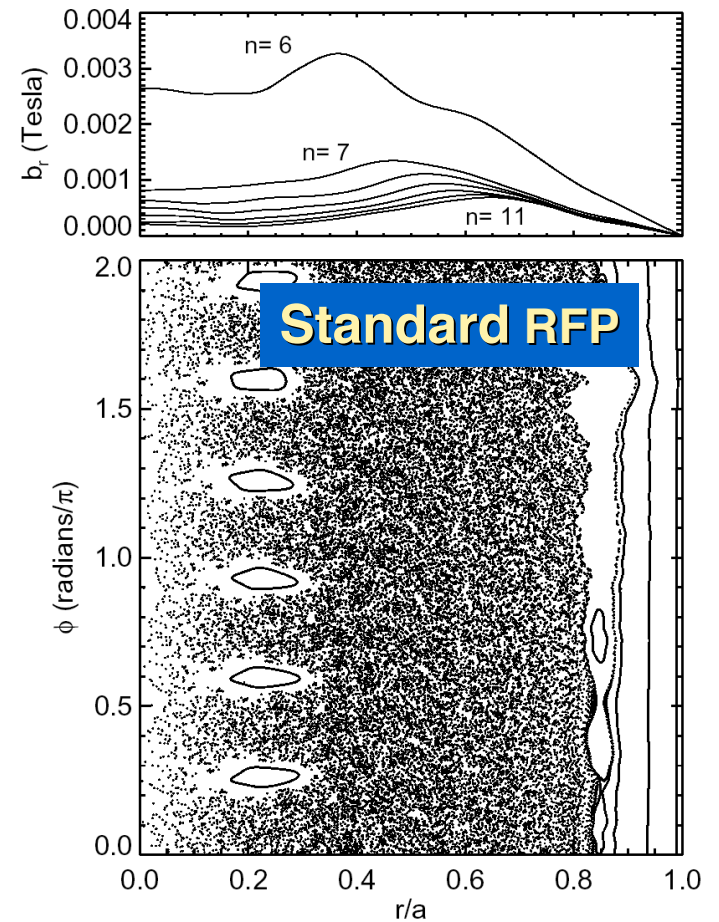
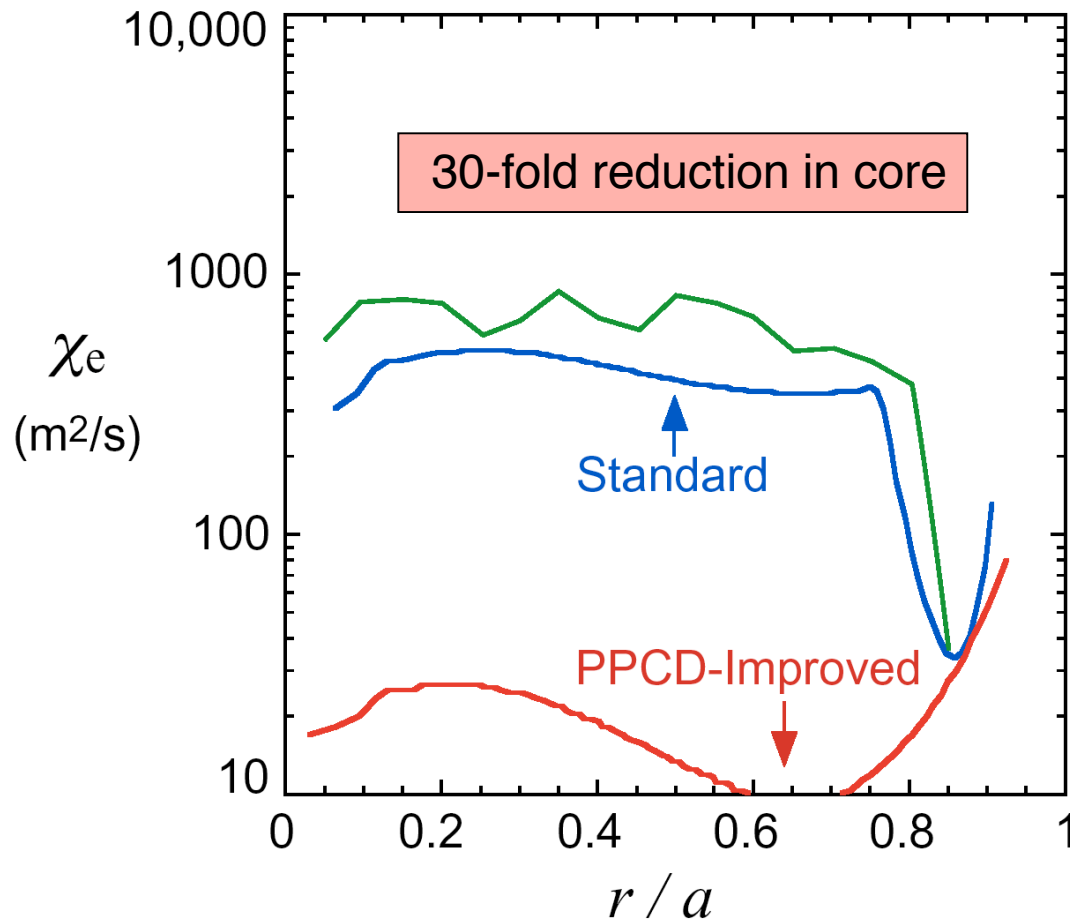
- Field line tracing used to directly evaluate  $D_m = \langle (\Delta r)^2 \rangle / \langle L \rangle$ .



3D resistive MHD computation  
 $S = 10^6, R/a = 3$   
 expt. measured  $\chi(r)$  profile

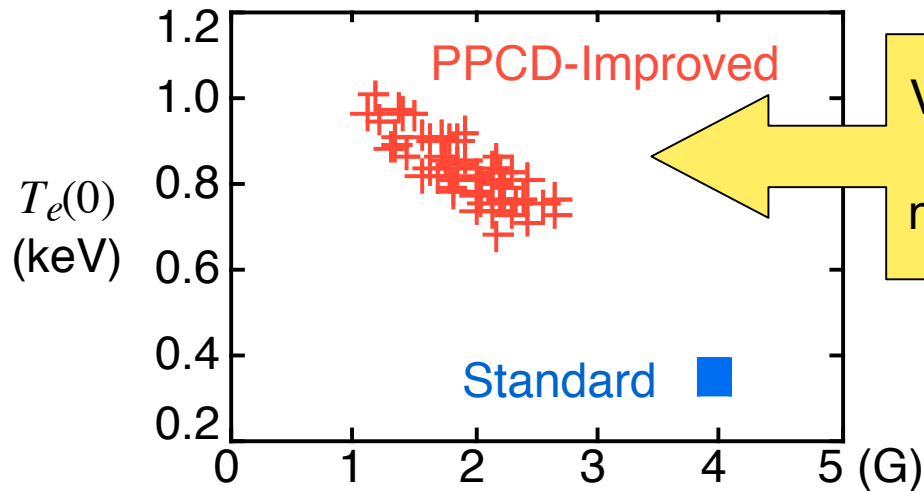


# Heat conductivity reduction greatest where stochasticity is usually most intense.



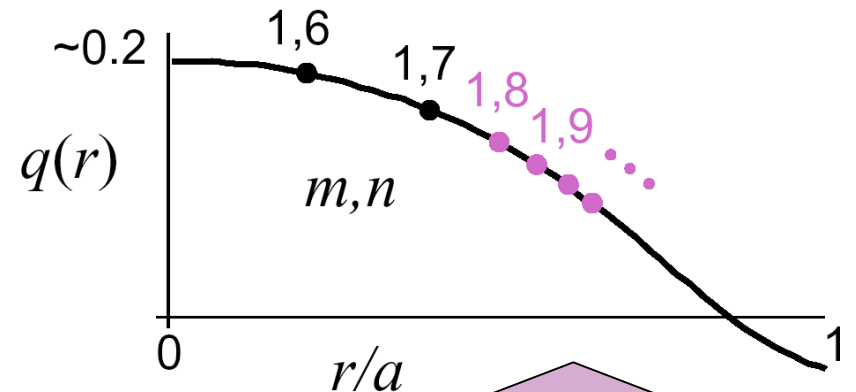
# Sustained reduction of mid-radius resonant modes leads to highest $T_e(0)$ , therefore largest $\beta_E$ .

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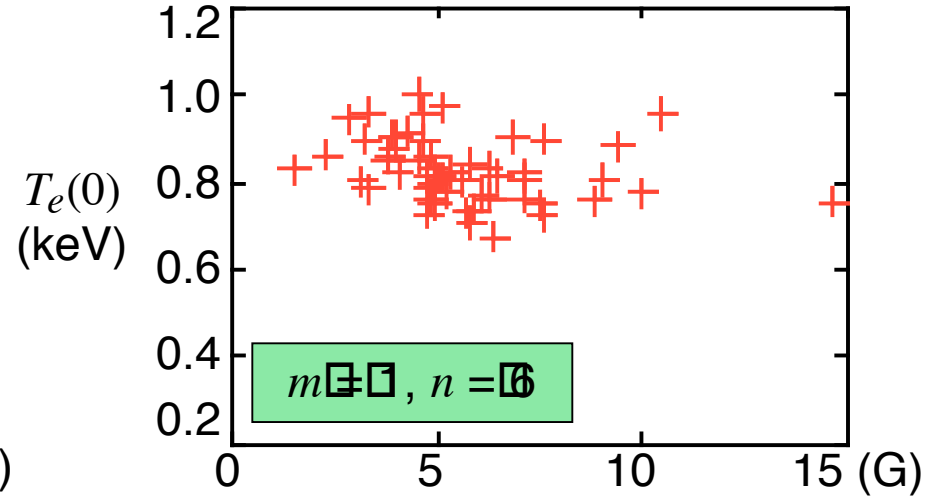
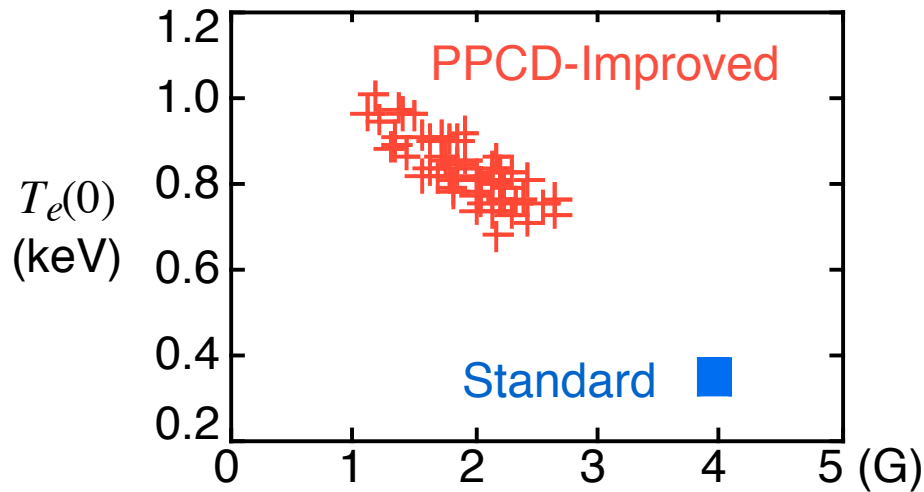
Variable fluctuation reduction reveals mode dependence.

Time Average  $b_{rms} = \sqrt{\sum_{n=8}^{15} b_n^2(a)}$



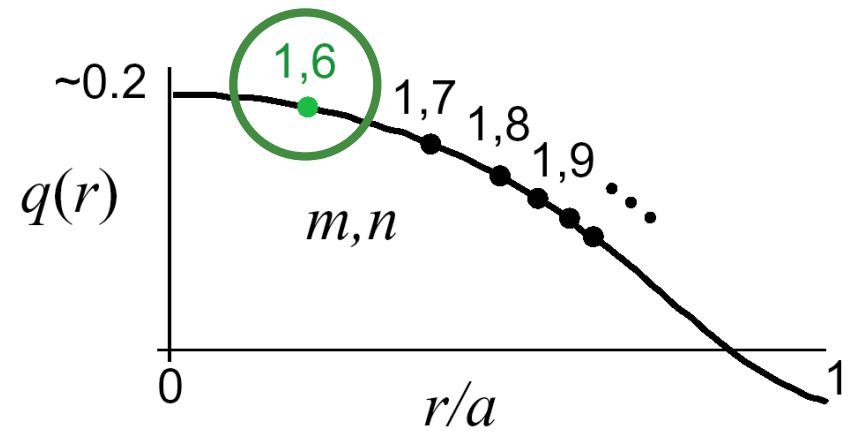
mid-radius modes  
 $m = 1, n \geq 8$

# $T_e(0)$ weakly correlated with dominant core mode.



Time Average  $b_{rms} = \sqrt{\sum_{n=8}^{15} b_n^2(a)}$

Time Average  $b_{1,6}$

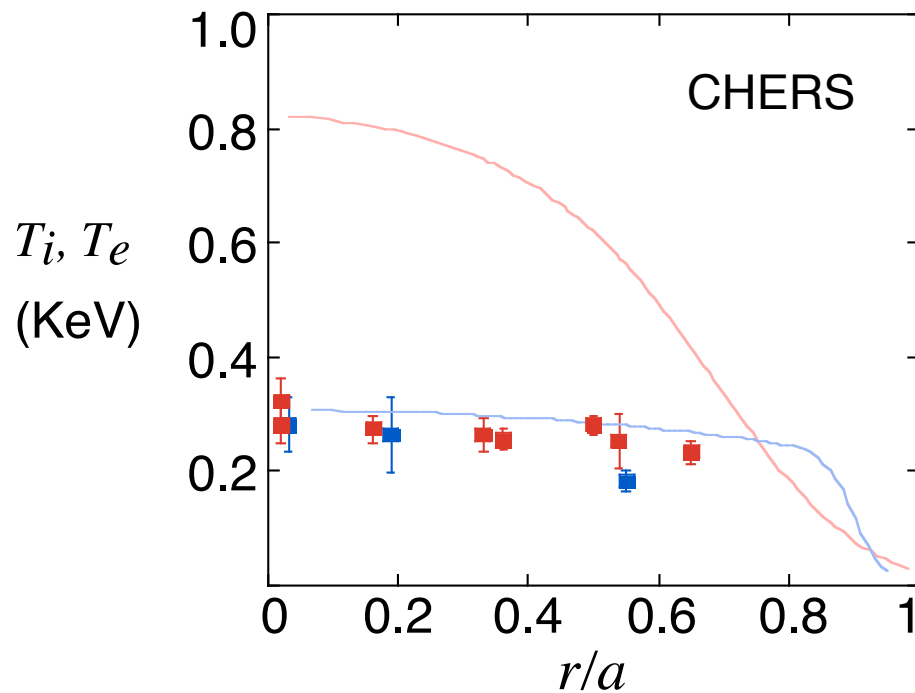


Broadband mode suppression key to improved RFP confinement.

# The ion temperature doesn't change during PPCD.

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- **Standard:**  $P_{e-i} \leq P_{CX}$  ;  $T_i / T_e > 0.5$  (>1 at sawtooth crash) □ anomalous ion heating
- **PPCD:**  $P_{e-i} \geq P_{CX}$  ;  $T_i / T_e < 0.5$



**Stochastic Transport**

$$\chi_i^{R/R} = \sqrt{\frac{T_i m_e}{T_e m_i}} \chi_e^{R/R}$$

Standard  
 $\sim 10 \text{ m}^2/\text{s}$  core  
 $\sim 1 \text{ m}^2/\text{s}$   $q = 0$

PPCD  
 $\sim 0.3 \text{ m}^2/\text{s}$

**Classical**

$$\chi_i^{classical} \sim \chi_i^2 / \chi_i \sim 0.1 \text{ m}^2/\text{s}$$

# Tools to extend plasma control in development.

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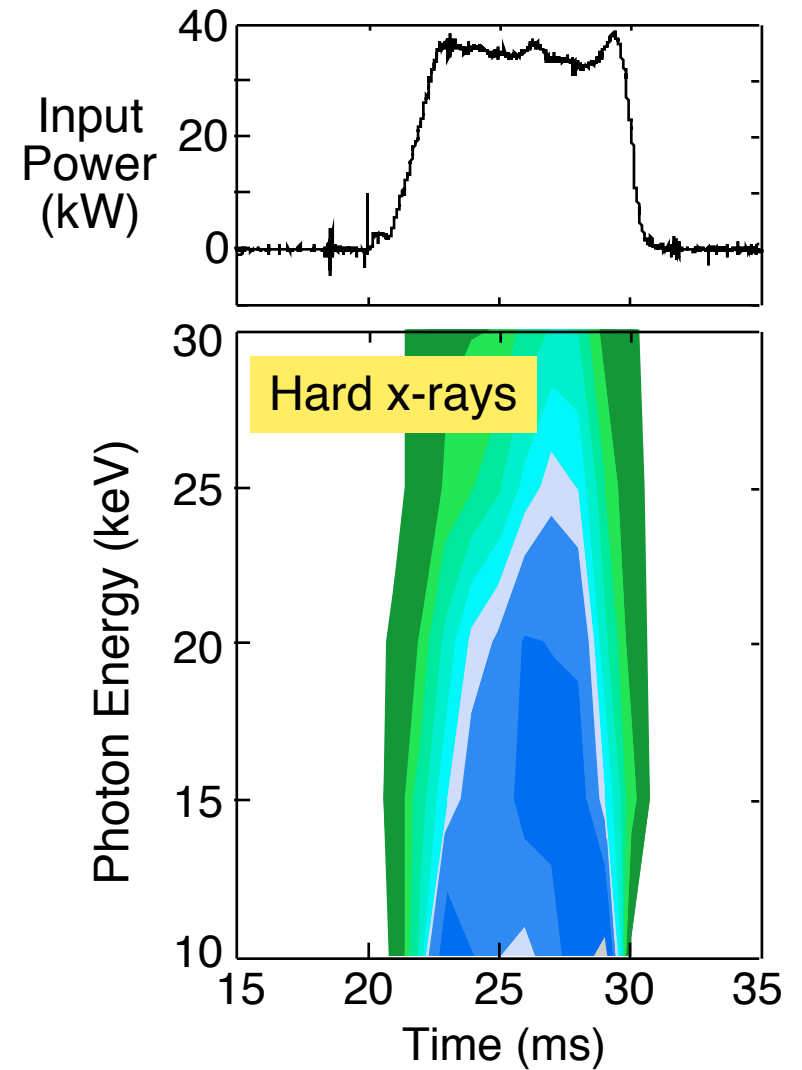
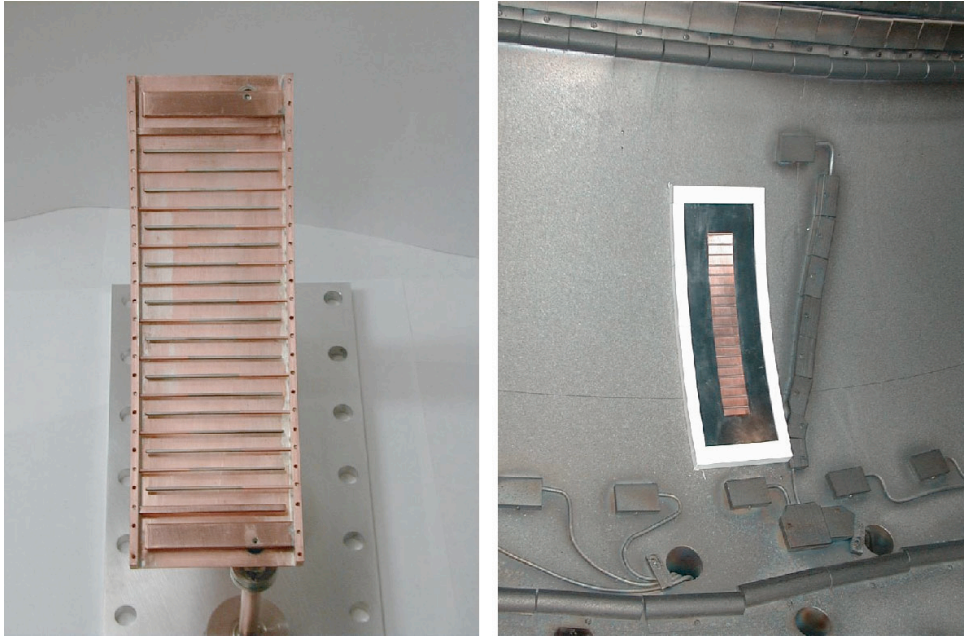
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- RF current drive for precise, localized  $J(r)$ -control and auxiliary heating:
  - Lower-hybrid (800 MHz,  $n_{\parallel} = 8$ , stripline antenna)
  - Electron Bernstein wave ( $\sim 3.5$  GHz, waveguide antenna)
  - Required power  $\sim 1$ -2 MW both waves (LH theoretically more efficient)
- Oscillating Field Current Drive for DC current sustainment (and possibly profile control) from purely AC inductive loop voltages (AC helicity injection).
- Neutral beam heating, e.g., investigate beta limit ( $P_{\square} < 1$  MW during PPCD).

# Second generation lower-hybrid antenna installed.

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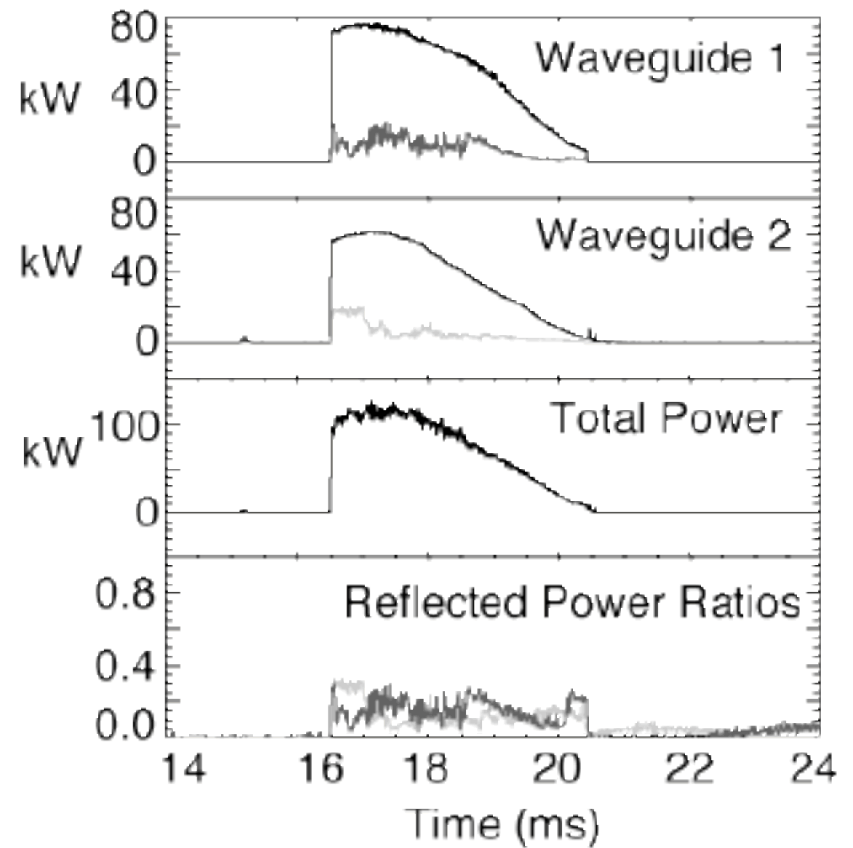
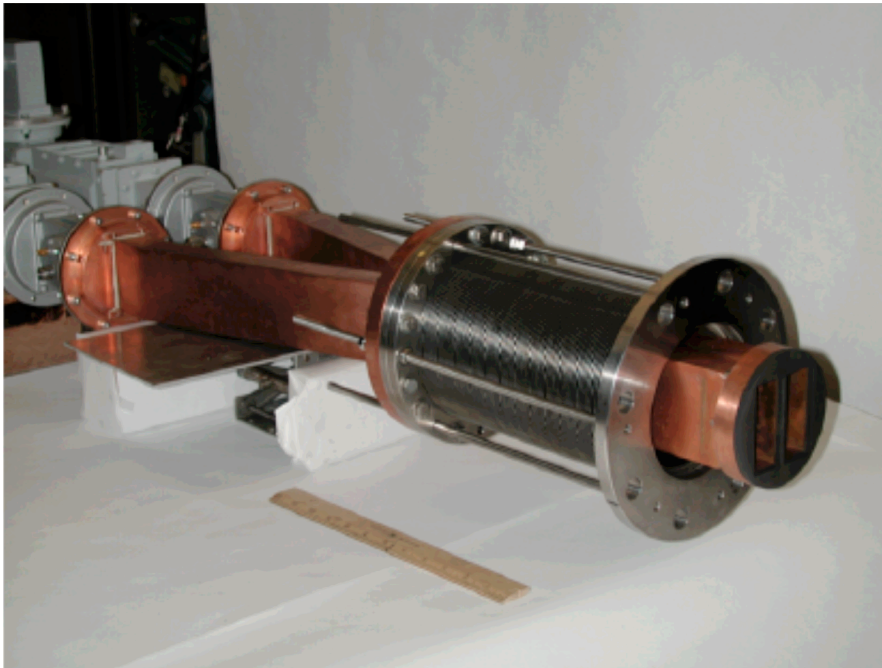
- Interdigital line antenna, 800 MHz,  $n_{||} = 8$



# Twin-waveguide installed to drive EBW in overdense RFP

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- 3.2-3.7 GHz (variable) from two 60 kW traveling wave tube sources.

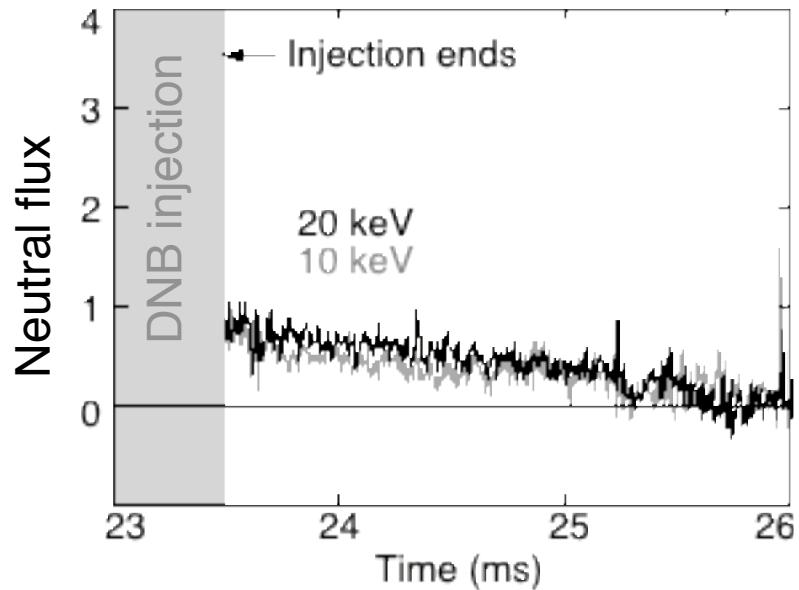




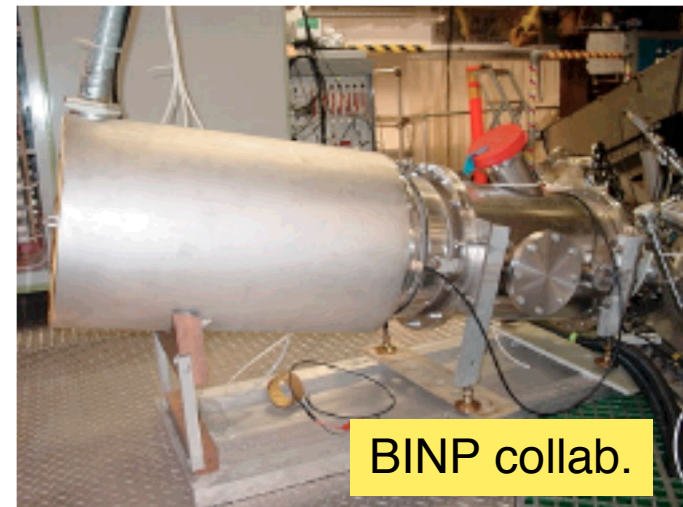
# Good fast ion confinement encouraging for NB heating.

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Fast neutral-decay following DNB



1 MW short pulse next step



# Summary.

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- Two paths for enhanced RFP energy confinement, with one common goal — reduced transport from magnetic stochasticity
- Tokamak-like energy confinement demonstrated at high beta and 10X smaller  $B(a)$  in MST with (transient) edge inductive current drive:
  - $\chi_e \sim 5 \text{ m}^2/\text{s}$  and  $\chi_{total} \leq 15\%$
  - Fast electrons confined, with velocity-independent diffusivity
- Broadband mode reduction key to improved confinement, but one remaining large mode is tolerable (analogous to tokamak sawtoothing)
  - Bodes well for “single-helicity” dynamo ... maybe even for multiple-helicity dynamo if high- $n$  scaling is strong
- Extending plasma control
  - RF, neutral beam, and OFCD tools in development (low power)
  - PPCD: find the optimum  $\mathbf{E}(a,t)$ , evaluate pulsed-reactor scenario